A REVIEW ON THE DEFENSE-IN-DEPTH CONCEPT AND THE FLEX STRATEGIES IN DIFFERENT COUNTRIES AFTER FUKUSHIMA ACCIDENT

by

Hong XU^{1*}and Baorui ZHANG²

¹ Jinyuyun Energy Technology Co., Ltd., Chongqing, China ² Institute of Nuclear and New Energy Technology, Tsinghua University, Beijing, China

> Review paper https://doi.org/10.2298/NTRP210128013X

To enhance the defense in depth for nuclear safety after the Fukushima nuclear accident, the U.S. Nuclear Energy Institute put forward the concept of diverse and flexible coping strategies and the corresponding FLEX support guidelines for the special scenarios of extended loss of alternating current power and loss of ultimate heat sink caused by beyond-design-basis external event. Subsequently, the idea of the FLEX strategy was broadly accepted and spread widely. The introduction of the concept of FLEX strategy into the defense in depth was the biggest improvement for nuclear safety in the recent decade. This paper has reviewed the concept of traditional defense in depth and its weakness that led to the Fukushima nuclear accident, which led to the development motivation for the FLEX strategy. The research progress of the FLEX strategy in different countries in the past ten years has been reviewed. Based on the literature, and the aforementioned review, some recommendations for future work have been presented.

Key words: diverse and flexible coping strategy, defense in depth, loss of ultimate heat sink, beyond-design-basis external event, extended loss of alternating current power, Fukushima nuclear accident

INTRODUCTION

According to the prediction of the International Atomic Energy Agency (IAEA), the demand for global primary energy will be an increment of about 2.5 times in 2050 comparing with the beginning of this century in 2000. Nuclear energy has the potential to contribute to a sustainable solution for the world's growing energy needs and also environmental problems [1]. After a slight decrease in nuclear generation around the Fukushima nuclear accident in 2011, the nuclear generation increased successively these years. The development of the world's nuclear industry currently faces economic, environmental, and safety concerns. The root of the concerns is nuclear safety [2].

Similar to most engineering systems, the safety of nuclear power plants (NPP) is the most critical concern. Nuclear engineering design is committed to maximizing nuclear safety. The NPP have been designed to withstand a large series of postulated initiating events, including design basis accidents (DBA) and design extension conditions (DEC) [3]. These two accidents may lead to severe accidents (SA) if they are not handled properly [4]. The DBA are postulated to establish the design bases of the safety systems, which are considered on the licensing basis. Representative DBA are main steam line break, loss-of-coolant accident, and so on [5]. The DEC were introduced with the purpose to further improve safety by enhancing NPP capability to withstand the conditions generated by accidents that are more severe than DBA [6]. Examples of DEC are station blackout, loss of ultimate heat sink (LUHS), and anticipated transient without scram [7].

To get the NPP license from the authority, the NPP design of the licensee offers a demonstrated protection using various safety and non-safety systems with application of emergency operating procedures (EOP) and severe accident management guidelines (SAMG) (if the EOP are not effective). The set of EOP and SAMG is designed based on scenarios, often using the deterministic thermal-hydraulic assessment [8, 9] and the probabilistic safety analysis (PSA) [10]. Based on the advanced design and improvement of safety procedures/guidelines, the core damage frequency and large early release frequency, which are the key safety criteria for NPP safety, became smaller and smaller.

^{*} Corresponding author; e-mail: xuhong2005@tsinghua.org.cn

According to the safety requirements, the current NPP design can prevent core damage under DBA or DEC conditions. However, core damage can occur under beyond-design-basis external events (BDBEE), especially extremely severe external events like the east Japan great earthquake that led to the Fukushima nuclear accident and subsequent large radiation release [11]. To enhance the defense in depth (DID) for nuclear safety after that, the U.S. Nuclear Energy Institute (NEI) put forward the concept of diverse and flexible coping strategies (FLEX) for the special purpose of BDBEE hazard mitigation and the corresponding FLEX support guidelines (FSG). Subsequently, the idea of the FLEX strategy was broadly accepted and spread widely in the recent decade. Since 2021 is the 10th anniversary of the Fukushima nuclear accident, the current progress, and the existing challenges of the FLEX strategy deserve a review, which is the main objective of this article.

DEFENSE-IN-DEPTH CONCEPT AND THE INTRODUCTION OF THE FLEX STRATEGY

This section presents a review of the concept of traditional DID and its weakness that led to the Fukushima nuclear accident, consequently the introduction of the FLEX strategy to enhance the DID for nuclear safety.

Defense in depth

Traditional DID

The main pillars for nuclear safety are based on the following concept as shown in fig. 1. At the begin-

ning of this century, the IAEA published its revised fundamental safety principles, which importantly states that *the fundamental safety objective is to protect people and the environment from harmful effects of ionizing radiation* [12]. Consequently, the concept and methodology of DID were brought out at the end of the last century [13], and then it spread to regulatory agencies around the world.

Except for building four physical barriers [14] as shown in fig. 1 for accident prevention or mitigation, the fundamental idea for DID is to set several consecutive and independent levels of protection to prevent or mitigate the consequences of accidents [15]. The NPP conditions are divided into five levels – normal operation condition, AOO, DBA, SA, and post-SA situation - for different protections [16]. The definition of each condition could be found in the IAEA report [17]. The frequencies from AOO to post SA situation are decreased from around 10^{-2} per reactor-year to less than 10^{-6} per reactor-year. If one level of protection or barrier is to fail, the subsequent level or barrier would be available.

The concept of DID could be well understood in fig. 2. Each blue board represents a defense layer for hazard prevention. The ideal condition of DID is shown in fig. 2 as the case (a). They are perfect without any failure and therefore there is no possibility of system loss. But case (a) is unrealistic, which only exists in theory [18]. A more realistic situation is shown in fig. 2 as the case (b). There may be several holes on each blue board, which means failures in each layer of DID. The failures in the first layer may be prevented by the second layer. If both the first layer and the second layer are failures, losses may be prevented by the third layer, and so on, the failures are shown as lines-1 in fig. 2 case (b). According to the PSA, the probability of failure with



Figure 1. Nuclear safety concept



Figure 2. Traditional DID concept for hazards

more DID layers will decrease sharply when the layers increase [19]. Consequently, there is nearly no chance for all layers of failure based on the independence assumption of the layers.

Weakness of traditional DID - cliff-edge effect

However, the Fukushima nuclear accident showed some defects of the traditional DID for implementation and highlighted the possibility that extreme natural phenomena could challenge the prevention, mitigation, and emergency preparedness of the DID layers [20]. From the perspective of review currently, this seemingly perfect DID theory has a significant weakness that DID does not consider the cliff edge (CE) which leads to the so-called cliff-edge effect (CEE) in the viewpoint of probability theory [21]. The CE could be divided into two types, physical CE and knowledge-oriented CE, which will be defined and discussed separately as follows.

- according to the definition of [22], the physical CE represent the phenomenon that there occurs a significant increase of consequence due to a small amount of decrease of the occurrence frequency of the external event. This definition of CE can be understood as depicted in the risk curve, *i. e.*, a relationship between an occurrence probability and its consequence, as shown in fig. 3. An example of the physical CE is the common cause failure (CCF) brought by large external events [23]. To make matters worse, if the external event is BDBEE, it may impact multi-units in the same site or NPP in the same disaster area [24].
- Knowledge-oriented CE is associated with the knowledge limit within which we can deduce and make reliable decisions based on our certain amount of knowledge and available information of an objective of interest. These CE imply the deviation from the known domain to the unknown



Figure 3. The CEE [22]

domain or phenomena unexpected. As an example, failure of containment may happen due to the unknown detailed mechanism of hydrogen detonation. As a result of knowledge-oriented CE, it may lead to inappropriate decisions, which may cause the failure of DID [25].

The concept of DID would be ineffective owing to CEE, as shown in fig. 4. The CCF (a kind of physical CE) may lead to failure of all layers of defense simultaneously or successively and consequently, leads to the failure of DID as shown in fig. 4(a). In fig. 4(b), although the hazard does not cause the original DID to fail completely, it may skip some layers of the DID system, as an example, the last layer in fig. 4(b), and as a result, lead to an unknown failure of the system.

Failure of DID in the Fukushima nuclear accident

To prevent such kinds of the accident like the Fukushima nuclear accident in the future, we need a way to reduce and avoid this kind of CE and enhance the concept of DID. After Fukushima nuclear accident, IAEA held a conference, which focused on DID issues and came to a summary about DID including:



Figure 4. Failure of DID concept owing to CEE

- The DID has to be strengthened and extensively applied in order to meet the most recent safety objectives,
- further development and guidance are required for the strengthening measures, and
- criteria to choose between fixed and mobile equipment should be developed.

To prevent or mitigate the hazard caused by BDBEE, additional strategies were proposed for coping with these events including the utilization of portable equipment, permanent equipment, or combination of portable and permanent equipment. The FLEX strategy focuses on maintaining, enhancing, or restoring NPP key safety functions that address the potential consequences of these BDBEE. Cavaluzzi has used the PSA method to prove that FLEX strategies with portable equipment could decrease the possibility of failure during BDBEE [26].

The introduction of the FLEX strategy

The kind of natural disaster that caused the Fukushima nuclear accident was not considered plausible in the vicinity of any NPP in the USA. But the U.S. NRC established a near term task force (NTTF) to determine whether safety improvements should be recommended for US commercial NPP, which finally issued a report [27] including 12 recommendations in total. Apart from laws and formal regulations, this report named *Recommendations for Enhancing Reactor Safety in the 21st Century: The Near-Term Task Force Review of Insights from the Fukushima Daiichi Accident* may be the single most influential document in the U.S. NRC history [28]. According to the different urgencies of the recommendations, the tasks were divided into 3 tiers as shown in fig. 5 [29].

In addition, many other organizations investigated the Fukushima nuclear accident after it happened, producing quite a lot of reports [30, 31]. Several researchers have reconsidered the lessons from Fukushima nuclear accident, which have been summarized here, as shown in fig. 6 [32, 33].

In the following year of the Fukushima nuclear accident, a further recommendation from NTTF was the U.S. NRC order EA-12-049 (issued on 12 March 2012) [34], requiring all U.S. NPP to implement mitigation strategies to protect against the scenario of Extended Loss of Alternating current (AC) Power (ELAP) caused by BDBEE. Consequently, U.S. NEI put forward the concept of FLEX and corresponding FSG specifically for external scenarios as the Fukushima nuclear accident, which use on-site or off-site replacement and substitute equipment, making connections and repairs, just like restoring power in any system [35]. Actually, the FLEX strategy may be considered as an extension of extensive damage mitigation guideline (EDMG), which was developed by U.S. NEI aiming at preventing or mitigating hazards caused by explosions or fires (especially these caused by man-made hazards such as aircraft attack) after the 911 terrorist attacks [36, 37]. Additionally, the concept of the FLEX strategy to use portable equipment to mitigate the accidents has already been adopted by IAEA (SSG-54), to deal with accidents, especially SA[38].

The strategy has been modified from draft revision 0 in 2012 to version 4 in 2016 [39] owing to its importance and urgency. In the last version of the NEI FLEX strategy report, the theory of FLEX strategy was supplemented with the shutdown modes analysis to identify and reduce the risk of showdown process, the *Appendix E* for



Key technical characteristics	 Insufficient consideration of CCF Underestimation of BDBEE (LOLA, ELAP, LUHS) Underestimation of the risk for multi-unit failure Lack of strategies for SFP in SA Lack of flexible equipments Insufficient PSA level 2 analysis 	Figure 6. Lessons from Fukushima accident
NPP accident management	 Shortage of information or wrong information Lack of procedures and training for SAMG Weak capabilities of improvisation/flexible strategies Lack of capabilities under loss of control room Human reliability assessment (HRA) 	

validation guidance of FLEX strategy, the *Appendix G* and *Appendix H* for mitigating strategies assessment of new flood and seismic hazard information, respectively.

Based on the identified accident sequences following the BDBEE and CE at each NPP, the objective of the FLEX strategy is to improve the resilience and flexibility for prevention and mitigation strategies of NPP, and consequently enhancing the DID [40]. The concept of the FLEX strategy was accepted by the U.S. Nuclear Regulatory Commission (NRC) in August 2012 [41]. According to the afore-mentioned recommendations from literature and issues for the Fukushima nuclear accident, and the characteristic of the FLEX strategy, the capabilities of the FLEX strategy could be summarized here as shown in tab. 1.

The aim of the FLEX strategy implemented during the BDBEE is to decrease the failure risk of NPP as low as possible. To achieve this objective, the FLEX strategy should be implemented to achieve the follow-ing functions [42]:

- mitigation of the remaining residual risks (*i. e.*, the risks caused by physical CE (CCF), and
- prevention and/or mitigation of the unknown risk,
 (*i. e.*, the risks caused by knowledge-oriented CE).
 Therefore, how the FLEX strategy enhances the

concept of DID could be explained by fig. 7.

RESEARCH PROGRESS OF FLEX STRATEGY IN DIFFERENT COUNTRIES

After the Fukushima accident, the responses of different countries are quite different from each other in the aspect of government policy and utilities post-accident measures, as different countries have

Category	Subcategory	Content	
	CCF	CCF for Multi-unit NPP site	
	Initial BDBEE	- Seismic events	
		– Flood	
		- High winds	
		- Extremely low temperatures (including snow, ice)	
		- Extremely high temperatures	
		- LOLA	
Technical strategy	Induced DEC	- ELAP	
		– LUHS	
	Prevention and mitigation capability for SFP accident	– SFP makeup	
		– SFP spray	
		– SFP leak mitigation	
	Phenomena of SA	- Containment venting	
	Enhance NPP procedures/guidelines	Strengthening and integration of NPP EOP / SAMG strategy for beyond design basis accidents (BDBA)	
Management strategy	Personnel management	– Staffing	
		– Training and drilling	
		- Communications	
		– Human reliability assessment	
	Resource management	On-site and off-site resource protection and deployment	



Figure. 7. The FLEX strategy enhanced DID concept

different national conditions and attitudes towards nuclear energy [28]. However, many countries now have decided to have portable equipment available for added capability, and there is considerable worldwide interest and research effort directed toward FLEX strategy, especially in the countries with high-density NPP sites such as Korea, since the possibilities of BDBEE and its hazards at multi-unit sites are larger. In this section, introductions and comparisons of the post-Fukushima safety enhancement measures related to FLEX (or similar to the concept of FLEX but with different nomenclatures) in different countries are described according to the available literature. It should be noted that even if similar concepts or actions are adopted in different countries, they have different nomenclatures. Additionally, this section mainly focuses on the regulatory requirements in observed countries and the response from their nuclear industries briefly since the FLEX strategy is site-specific and different NPP developed their own detailed strategies, which are dissimilar from each other. For more detailed information about the FLEX strategies at a specific NPP site, the corresponding reference can be resorted to.

USA

Following the Fukushima accident, the U.S. NPP assessed the safety items and concluded that BDBEE (*e. g.*, seismic events, external flooding, *etc.*) are highly unlikely but could present challenges to NPP. The Fukushima Response Steering Committee, a leadership structure formed to integrate and co-ordinate the industry's ongoing response to the Fukushima nuclear accident, developed the FLEX concept. The committee – senior electric utility executives, reactor owners'

groups, the Nuclear Energy Institute (NEI), the Institute of Nuclear Power Operations, and the Electric Power Research Institute-spent the year following Fukushima ensuring that its lessons are fully understood and integrated into plans to enhance safety.

Numerous industry activities related to NPP procedures/guidelines were initiated to implement lessons learned in the USA [43]. In addition to updating the generic EOP/SAMG strategy, the usage of EDMG and FLEX equipment has attracted much attention [44]. The integration of the updated EOP/SAMG strategy with EDMG, FSG, and emergency mitigating equipment guidelines (EMEG) is an essential part of the activities.

After the FLEX strategy had been developed by NEI, all U.S. NPP had to develop FLEX strategies to protect against ELAP resulting from BDBEE and submit an *overall integrated plan*. These so-called FLEX integrated plans should have had to be submitted to the US regulator no later than two refueling cycles after submittal, or by end of 2016, whichever came first [45].

Under the FLEX program, NPP owners have invested heavily in additional on-site diesel generators and diesel-driven pumping systems. Efforts have been made to expand on-site diesel fuel storage capabilities. Several FLEX integrated plans could be resorted to for more detail, for example:

- Nuclear Innovation North America (NINA) reviewed the FLEX capabilities of the NPP at the South Texas Project Units 3 and 4 (STP3&4), which are the US-Advanced Boiling Water Reactors (ABWR) and concluded that the US-ABWR was capable of providing a significant coping period for an ELAP and consequent LUHS without core damage by using existing plant systems and considering also FLEX strategy [46].

- Fort Calhoun NPP established detailed flow charts in procedures/guidelines to introduce the FSG for different safety objectives in ELAP/LUHS scenarios. Simultaneously, a list of FLEX strategies was built to modify the *support optimal strategies* [47].
- Palo Verde NPP has checked the safety issues after the Fukushima nuclear accident and introduced the FLEX equipment, such as portable pumps, FLEX generators, condensate storage tanks, and the refueling water tank to enhance the safety systems in NPP [45].
- The 3-unit Browns Ferry NPP Authorized by the Tennessee Valley has the capacity to store at least 282, 240 gallons of diesel fuel on-site for its FLEX diesel generators [48].

Finally, the US NRC intended to issue a Safety Evaluation Report for each site following the demonstration of successful implementation of the FLEX plan.

Based on the summary of feedback experience and research on FLEX strategies after the Fukushima nuclear accident, the U.S. NRC proposed to amend Title 10 of the Code of Federal Regulations (10 CFR). The new rules were finally issued in August 2019 [49], with the following emphasized key points and requirements of FLEX strategies for each applicant or licensee:

- build the integrated response capability that includes FSG, EDMG, and EOP,
- develop, implement, and maintain a supporting organizational structure with defined roles, responsibilities, and authorities for directing and performing the FSG and EDMG,
- develop, implement, and maintain sufficient staffing to support the implementation of FSG and EDMG in conjunction with the EOP during an event, and
- provide training, drills, or exercises to personnel that perform activities in accordance with FSG and EDMG.

The SAFER (short for Strategic Alliance for FLEX Emergency Response) team, an alliance established between AREVA and Pooled Inventory Management, is contracted by the U.S. nuclear industry to establish and operate National SAFER Response Centers to purchase, store, maintain and deliver emergency response equipment in the case of a major nuclear accident or BDBEE in the U.S. [50]. Two SAFER control centers were built in Lynchburg and Birmingham separately. Additionally, two regional response centers Memphis and Phoenix (also called FLEX support centers) have been built with equipment, logistic, and support technicians for the deployment of off-site FLEX strategies within 24 hours [51].

The NRC also emphasizes that it is allowed for a licensee to make changes to FSG and EDMG without prior NRC approval, provided that the licensee performs an evaluation demonstrating that regulatory requirements continue to be met. Documentation of all changes would need to be maintained.

In addition, the U.S. NRC has made efforts to credit FLEX strategies in regulatory applications [52], such as investigating human reliability assessment methods for the FLEX context [53], incorporating the FLEX equipment into PSA [54] and overseeing the licensee's implementation of FLEX strategies, preparing Regulatory Guide 1.226 to guide the licensees on how to demonstrate their compliance with regulations of BDBEE planning and preparedness.

Canada

Following the Fukushima nuclear accident, Canadian NPP procured equipment and initiated modifications to improve response capability for BDBA. Consequently, modified guidance was correspondingly introduced for these changes to address BDBA [55]. To prevent SA, emergency mitigating equipment are introduced as additional barriers for accident management, maintaining reactor core cooling, and protecting the integrity of containment in Canada. Hence, EMEG, which is similar to EDMG and FLEX, is being prepared in parallel with enhancements of SAMG by reflecting the lessons learned from the Fukushima nuclear accident [56].

From then on, the Canadian Nuclear Safety Commission regulatory document REGDOC-2.5.2 [57] requires the design authority to consider mitigation of a broad range of accidents and provide the initial accident response guidance including EMEG, which is robust, readily available, easily deployable within required timeframes, and has adequate redundancy [58].

European Union

The Fukushima nuclear accident triggered the need for coordinated action at the EU level to identify potential further improvements in NPP safety. The Council of the EU asked the European Commission and the European Nuclear Safety Regulators Group (ENSREG) to investigate the robustness of a plant for events beyond its licensed design basis and reassess the safety margins of NPP in the light of the events in Fukushima [59]. On 25th March 2011, ENSREG decided that the safety of all EU nuclear plants should be reviewed, based on comprehensive and transparent risk and safety assessments – the stress tests [60]. The stress tests consist of three main steps: a self-assessment by licensees, followed by an independent review by the national regulatory bodies, and by the third phase of international peer reviews. The international peer review phase consists of three steps: an initial desktop review, three topical reviews in parallel, and seventeen individual country peer reviews. Through the stress test, each specific plant could identify its weak points and any CEE by the postulated extreme natural events, and could further find any provisions to prevent these CEE or increase its robustness through modification of hardware or procedures/guidelines, organizational preparedness, *etc.* The FLEX strategies in France and Spain will be introduced following as examples of the EU.

France

Following the Fukushima nuclear accident, the French Nuclear Safety Authority asked the French nuclear licensees to carry out a reassessment of their facilities in the light of the Fukushima nuclear accident. These reassessments, called Complementary Safety Assessments, were based on the specifications attached to the aforementioned decisions and consistent with the specifications for the stress tests requested by the European Council. With the assessment results, France has shown that its nuclear facilities have a satisfactory level of safety. However, it had been decided to significantly improve their robustness to extreme situations, by introducing nuclear rapid response force (FARN), which has a similar objective and function to EDMG. In addition, some risk of CEE has been identified during the assessments, corresponding measures, which was called hardened safety core (HSC), similar to the FLEX strategy in the USA, have been developed, providing a set of material to enable the NPP to withstand hazards or situations caused by the CEE [61]. The HSC must ensure ultimate protection of nuclear facilities with the following three objectives:

- Prevent a SA or limit its progression,
- limit large-scale releases in the event of an accident which it was not possible to control, and
- enable the licensee to perform its emergency management duties [62].

The HSC may be composed of existing structures, systems, and components (SSC), that might require to be strengthened, or of new SSC that should be designed and sized to withstand extreme situations. The SSC may be active or passive. The implementation of HSC for operating NPP compensated for some weaknesses in the current approach and improved significantly the robustness of the installations against BDBEE [63].

Spain

After Fukushima nuclear accident, two complementary technical instructions (ITC) were issued by the Spanish Nuclear Safety Council: ITC-1 according to the European *stress tests*, and ITC-2 about the potential LOLA of NPP due to big explosions and fires. Based on the assessments of *stress tests*, the Spanish NPP confirmed the robustness of the Spanish nuclear fleet and proposed a series of improvements aimed at reinforcing the response to BDBEE, thus increasing safety margins [38]. Both EDMG and FLEX strategies have also been established in Spain for hazards caused by BDBEE.

A new Alternative Emergency Management Centre (in Spanish, CAGE) has been built at each one of the Spanish NPP. These new centers are designed to allow the management of the emergency in LOLA scenarios, which are independent and have the resources to deal with the proposed scenarios autonomously for 72 hours, providing protection to the intended personnel [37]. The emergency support service of CAGE aims to strengthen the NPP emergency capabilities, by integrating with the Emergency Response Organization of NPP. In addition, a common Emergency Support Center (CAE) has been established, sharing resources (such as portable diesel generators, portable diesel pumps) among the different Spanish NPP and capable of providing support in the event of an emergency at any of the sites. The CAE service is available to any Spanish NPP. The service can be activated at any time of day and any day of the year. Once activated, the CAE acts under the instructions of the Director of the Emergency Plan of the NPP that activates the service. The CAE mobilizes equipment and personnel to the NPP site, to respond to the request made in less than 24 hrs. after service activation. The operation can be summarized in three sequential stages: activation, mobilization, and deployment [51]. Additionally, a logistics company Carreras Logistics Group is operative and available for supporting any Spanish NPP in an emergency [64].

Korea

Stress tests were required for all the NPP after the Fukushima nuclear accident in the Republic of Korea. Regarding the BDBEE such as Earthquake and Tsunami, their induced loss of safety functions (ELAP + LUHS) and possible severe accident, Korea had divided the stress tests into 3 steps:

- Operator self-assessment,
- adequacy review of including plant walk-down and detailed reviews of the regulator by experts, and
- reviews of nuclear safety and security commission, which was launched on October 26, 2011, as a regulatory body directly under the President in charge of strengthening independence and nuclear safety [65].

Based on the stress tests, a total of 56 post-Fukushima action items were considered to enhance nuclear safety, such as the modification of structure and equipment design against BDBEE, the reinforcement of emergency response, *etc.* [66]. A centralized expert team called Severe Accident Fast Response Expert Team has been built at Korea Hydro and Nuclear Power Company. The team can be dispatched to the emergency site within 6hrs. from the company Central Research Institute in Daejeon to any NPP with disaster in Korea [67].

Furthermore, several advanced and detailed studies have been done in Korea, such as the study on FLEX in a multi-unit site during a BDBEE case [41], integrating EDMG and FLEX to provide comprehensive strategies for NPP [68], the introduction of an integrated passive safety system to achieve various passive functions for FLEX strategies in OPR1000 NPP [69], the introduction of humanoid robotics for nuclear disaster management [70].

Based on the afore-mentioned study, Korea revised the nuclear law in Jun 2016 that before June 2019 all operating NPP should submit accident management plans (AMP), which included not only EOP and SAMG but also EDMG and MACST operating guidelines [68]. The MACST means a multi-barrier accident coping strategy that is similar to the FLEX strategy of the USA.

China

Immediately after the Fukushima nuclear accident, China's State Council decided to perform a comprehensive safety inspection of the operating NPP and NPP under construction, and suspend the construction permit issuance process for new NPP. China's State Council required that specific measures should be designed against man-made/natural DBDEE, for example, malevolent airplane impact and external flooding. Additionally, the FLEX strategy shall be available to instruct the actions of the operator in such cases. Consequently, several studies related to FLEX strategies have been done for different NPP types in China. During the period of AP1000 technical transfer from Westinghouse to China, Xu et al. focused on in-vessel retention strategy [71] and its uncertainty [72] in an assumed SA to enhance the DID of AP1000 safety system. Xu [73] has also studied the EDMG/FLEX strategy for the SFP of AP1000 by using the on-site existed system, fire system, and portable devices. Xing and Wang [16] have concentrated on the FLEX strategy the Hualong No. 1 (China's domestic for state-of-the-art Generation III NPP and the first unit of Hualong No. 1 in the world was put into commercial operation on January 30, 2021). In Hongyanhe NPP (CPR1000) [74], the EDMG and FLEX strategies have been established. Additionally, related to FLEX development in China, Xu et al. [75, 76] have concentrated on the procedures to develop FLEX strategy based on the summary of the literature. Yu et al. [77] have built an integrated strategy which was divided into three parts for control and command, control room recovery, and accident management separately.

FUTURE WORK RECOMMENDATION

The FLEX strategy was proposed after the Fukushima nuclear accident. Many related studies

could be found in the literature and it has developed significantly in the past ten years. But the current research is relatively preliminary and not systematic. It can be expected that there will be more FLEX strategy-related research in the future. Several research directions are proposed as follows:

- The BDBEE may impact all the plants on-site, multi-unit FLEX strategy needs to be considered in detail, not only from the aspect of technical issues but also the management.
- Some researchers began to consider the economical characteristic of the use of portable equipment/devices. But more detailed methodology needs to be established for quantitative analysis.
- Integration of the FLEX strategy has always been the focus of the FLEX strategy research, but so far it has not achieved a completed integration, and different countries have different understandings of integration. A more powerful strategy set for accident management needs to be built.
- A methodology for the on-site and *off-site* resource management (including the logistics) during a BDBEE scenario to improve their reliability should be developed to achieve a highly efficient intervention of the equipment/devices.

AUTHORS' CONTRIBUTIONS

Both of the authors made valuable contributions to this review work. H. Xu and B. Zhang both connected the literature, summarized the key points of each literature, and reviewed them. The manuscript was written by H. Xu with the help of B. Zhang.

ACRONYMS

ABWR	_	advanced boiling water reactors
AC	_	alternating current
AMP	_	accident management plans
AOO	_	anticipated operational occurrence
BDBA	_	beyond design basis accident
BDBEE	_	beyond-design-basis external event
CAE	_	emergency support center (in Span-
		ish)
CAGE	_	alternative emergency
		management centre
CCF	_	common cause failure
CE	_	cliff edge
CEE	_	cliff-edge effect
DBA	_	design basis accident
DEC	_	design extension condition
DID	_	defense-in-depth
EDMG	_	extensive damage mitigation guide-
		line
ELAP	_	extended loss of alternating
		current (AC) Power

EMEG	_	emergency mitigating
		equipment guideline
ENSREC	-í	european nuclear safety
		regulators group
EOP	_	emergency operating procedure
EROS	_	emergency response data system
FLEX	_	diverse and flexible coping strategy
FSG	_	FLEX support guideline
HRA	_	human relinbility assessment
HSC	_	hardened safety core
IAEA	_	International Atomic Energy Agency
ITC	_	complementary technical instructions (in
		Spanish)
LOLA	_	loss of large area
LUHS	_	loss of ultimate heat sink
MACST	_	multi-barrier accident coping
		strategy
NEI	_	U.S. Nuclear energy institute
NO-	_	normal operation
NPP	_	nuclear power plant
NRC	_	nuclear regulatory commission
NTTF	_	near-term task force
PSA	_	probabilistic safety analysis
RCS	_	rector core cooling system
SA	_	severe accident
SAFER	_	strategic alliance for FLEX
		emergency response
SAMG	_	severe accident management
		guideline
SBO	_	station blackout
SFP	_	spent fuel pool
SSC	_	structure, systems and components
		• •

REFERENCES

- Rachamin, R., *et al.*, Feasibility Assessment of the Once-Through Thorium Fuel Cycle for the PTVM LWR Concept, *Annals of Nuclear Energy*, 85 (2015), pp. 1119-1130
- [2] Xu, H., et al., Studies on the Criterion for Choking Process in Two-Phase Flow, Progress in Nuclear Energy, 133 (2021), 103640
- [3] Vayssier, G., Accident Management Under Extreme Events, *Journal of Energy*, 65 (2016), 2, pp. 21-31
- [4] Yoo, K. H., et al., Prediction of Golden Time Using SVR for Recovering SIS Under Severe Accidents, Annals of Nuclear Energy, 94 (2016), pp. 102-108
- [5] Thangamani, I., *et al.*, Thermal Stress Analysis of Containment Building in Case of a Main Steam Line Break (MSLB), *Nuclear Engineering and Design*, 239 (2009), pp. 1660-1672
- [6] Usmani, A., Aziz, T., Considerations in Implementing Earthquake Design Extension Condition (DEC), *Proceedings*, 11. International Conference on CANDU Maintenance and Nuclear Components; Toronto, Canada, 2017
- [7] Friend, M. T., et al., Simulated AP600 Response to Small-Break Loss-Of-Coolant-Accident and Non-Loss-Of-Coolant Accident Events: Analysis of SPES-2 Integral Test Results, *Nuclear Technology*, 122 (1998), 1, pp. 19-42
- [8] Xu, H., *et al.*, Sensitivity Analysis of Thermal-Hydraulic Models Based on FFTBM-MSM Two-Layer

Method for PKL IBLOCA Experiment, Annals of Nuclear Energy, 147 (2020), 107732

- [9] Xu, H., *et al.*, Optimization of the Nodalization of Nuclear System Thermal-Hydraulic Code Applied on PKL Benchmark, Journal of Nuclear Engineering and Radiation Science, 2021, (accepted)
- [10] Niehaus, F., Use of Probabilistic Safety Assessment (PSA) for Nuclear Installations, *Safety Science*, 40 (2002), 1-4, pp. 153-176
- [11] Kim, M. J., *et al.*, Radiological Safety Assessment for Incident-Free Transportation of Radioactive Decontamination Waste After the Fukushima Nuclear Power Plant Accident, *Nucl Technol Radiat*, 34 (2019), 2, pp. 201-208
- [12] ***, International Atomic Energy Agency, Fundamental Safety Principles, IAEA Safety Standards Series, 2006, Safety Fundamentals, No. SF-1, Vienna: International Atomic Energy Agency
- [13] ***, International Atomic Energy Agency, Defence in Depth in Nuclear Safety, International Atomic Energy Agency, 1996, Vienna, Austria, Report No. INSAG-10
- [14] ***, U.S. Nuclear Regulatory Commission, Historical Review and Observations of Defense-in-Depth. U.S. Nuclear Regulatory Commission, 2016, NUREG/KM-0009, ML16104A071, April 2016
- [15] Kim, H. J., Beyond Design Basis Accidents in Design Process, Colloquium at Department of Nuclear & Quantum Engineering, Kaist, Daejeon, South Korea, 2016
- [16] Xing, J., Wang, H., Practical Elimination on Large Release of Radioactive Materials and Safety Performance Research on HPR1000, *Proceedings*, 20th Pacific Basin Nuclear Conference, 2017, Springer Science and Business Media, Singapore, pp. 385-397
- [17] ***, International Atomic Energy Agency, Considerations on the Application of the IAEA Safety Requirements for the Design of Nuclear Power Plants, International Atomic Energy Agency, 2016, Vienna, Austria, Report No. IAEA-TECDOC-1791
- [18] Larouzee, J., Human Error and Defense in Depth: from the "Clambake" to the "Swiss Cheese", Resilience: A New Paradigm of Nuclear Safety, Springer, New York, USA, 2017, Cham, pp. 257-267
- [19] Fleming, K. N., Silady, F. A., A Risk Informed Defense-in-Depth Framework for Existing and Advanced Reactors, *Reliability Engineering & System Safety*, 78 (2002), 3, pp. 205-225
- [20] Miller, J. S., Preparing for the Extended Loss of AC Power (ELAP) Event in the USA, *Proceedings*, 22th International Conference on Nuclear Engineering (ICONE22), Prague, Czech Republic, 2014
- [21] Takada, T., et al., Development of Seismic Countermeasures Against Cliff Edges for Enhancement of Comprehensive Safety of NPP – Part 1: Conceptual Study on Identification and Avoidance of Cliff Edges of NPP Against Earthquakes, Proceedings, SMiRT-24, Busan, Korea, 2017
- [22] ***, U.S. Nuclear Regulatory Commission, A Proposed Risk Management Regulatory Framework, U.S. Nuclear Regulatory Commission, NUREG-2150, ML12109A277, April 2012
- [23] Čepin, M., Advantages and Difficulties with the Application of Methods of Probabilistic Safety Assessment to the Power Systems Reliability, *Nuclear Engineering and Design*, 246 (2012), pp. 136-140
- [24] Duy, T. D. L., et al., Probabilistic Safety Assessment of Twin-Unit Nuclear Sites: Methodological Elements, *Reliability Engineering & System Safety*, 145 (2016), pp. 250-261
- [25] Omoto, A., Where Was the Weakness in Application of Defense-in-Depth Concept and Why? Springer 2015,

Reflections on the Fukushima Daiichi Nuclear Accident, pp. 131-164

- [26] Cavaluzzi, J. M., Time-Based Risk-Informed Safety Margins: Concepts and Application to Heterogeneous Systems, M. S. Thesis (2015), Texas A&M University, College Station, Texas, Nuclear Engineering
- [27] Miller, C., et al., Recommendations for Enhancing Reactor Safety in the 21st Century: The Near-Term Task Force Review of Insights from the Fukushima Dai-Ichi Accident, United States Nuclear Regulatory Commission (2011), SECY-11-0093, ML111861807, Washington, DC, USA
- [28] Blake, E. M., The Response to Fukushima Daiichi in the United States, *Nuclear New*, 57 (2014), 2, pp. 27-29
- [29] Murata, A., Cultural Aspects as a Root Cause of Organizational Failure in Risk and Crisis Management in the Fukushima Daiichi Disaster, *Safety Science*, 135 (2021), 105091
- [30] ***American Nuclear Society, Fukushima Daiichi: ANS Committee Report, American Nuclear Society, La Grange Park, IL, March 2012
- [31] Koshizuka, S., Report from Investigation Committee on the Accident at the Fukushima Nuclear Power Stations of Tokyo Electric Power Company, *Journal of the Atomic Energy Society of Japan*, 54 (2012), 642
- [32] Kumar, M., et al., Post Fukushima Lesson Learned for Probabilistic Safety Assessment, Proceeding, European Safety and Reliability (ESREL) Conference, Zurich, Switzerland, 2015
- [33] Gajdoš, M., Practical Examples of SAMG from PWROG, Including Rules of Usage/TSC Guidelines. IAEA Workshop on the Development of SAMGs Using the IAEA's SAMG Development Toolkit, Vienna, Austria, 2017
- [34] ***, U.S. Nuclear Regulatory Commission, Issuance of Order to Modify Licenses with Regard to Requirements for Mitigation Strategies for Beyond Design Basis External Events, U.S. Nuclear Regulatory Commission, EA-12-049, ML16200A315, March 2012
- [35] Duffey, R. B., The Risk of Extended Power Loss and the Probability of Emergency Restoration for Severe Events and Nuclear Accidents, *Journal of Nuclear Engineering and Radiation Science*, 5 (2019), 031601
- [36] ***, Nuclear Energy Institute, B.5.b Phase 2&3 Submittal Guideline, Nuclear Energy Institute (2009), NEI 06-12 (Revision 3)
- [37] Caro, R. J., Enhancements to Emergency Preparedness and Response in Spain, *International Nuclear Safety Journal*, 5 (2016), 1, pp. 88-99
- [38] ***, International Atomic Energy Agency, Accident Management Programmes for Nuclear Power Plants, IAEA Safety Standards for Protecting People and the Environment (2019), Specific Safety Guide, No. SSG -54, Vienna: International Atomic Energy Agency
- [39] ***, Nuclear Energy Institute, Diverse and Flexible Coping Strategies (FLEX) Implementation Guide, Nuclear Energy Institute (2016), NEI 12-06 (Revision 5), ML18120A300
- [40] Schmitt, K. A., Function Allocation in Complex Socio-Technical Systems: Procedure Usage in Nuclear Power and the Context Analysis Method for Identifying Design Solutions (CAMIDS) Model, Ph.D. thesis Florida Institute of Technology Melbourne, Fla., USA, 2013
- [41] Jee, M. H., Kim, H. T., Basic Approach of Korean EDMG and FLEX Development, Transactions of the Korean Nuclear Society Spring Meeting, Gwangju, Korea, 2013
- [42] Prošek, A., Volkanovski, A., Extended Blackout Mitigation Strategy for PWR, *Nuclear Engineering and Design*, 295 (2015), pp. 360-373

- [43] Labarge, N. R., et al., Insights from Development of the Combined PWR SAMG, International Topical Meeting on Probabilistic Safety Assessment and Analysis, Columbia, SC, 2013
- [44] O'Brien, J., Beyond Design Basis Event Pilot Evaluations at US Department of Energy Nuclear Facilities, *Transactions of American Nuclear Society*, 108 (2013), 1, pp. 562-564
- [45] Powell, M., et al., The Impact of FLEX on Outage Risk, Nuclear Energy Institute Magazine, June, 38-40, 2014
- [46] Arai, K., et al., US-ABWR Design Features and FLEX Concept for Extended Loss of AC Power Events, Mechanical Engineering Journal, 4 (2017), 2, 15-00614
- [47] ***, Omaha Public Power District (OPPD), Fort Calhoun Station Diverse and Flexible Coping Strategies (FLEX) Overall Integrated Plan. U.S. Nuclear Regulatory Commission (2013), LIC-13-0019, ML13116A279
- [48] Greene, S. R., Nuclear Power and Electric Grid Resilience: Current Realities and Future Prospects, Ph.D. thesis University of Tennessee, Knoxville, Tenn., USA, 2018
- [49] ***, U.S. Nuclear Regulatory Commission, 10 CFR Parts 50 and 52 Mitigation of Beyond-Design-Basis Events, Federal Register, 84, 154, RIN 3150-AJ49, August 2019
- [50] Harter, R., Beyond Design Bases Event Response: FLEX, *Proceedings*, IAEA Workshop on the Development of SAMG Using the IAEA's SAMG Development Toolkit, Vienna, Austria, 2017
- [51] Caro, R. J., CAE The Spanish Emergency Support Center: A Centralized and Shared Emergency Support Service for Beyond Design Basis Events, *Journal of Nuclear Engineering and Radiation Science*, 2 (2016), 4, 044504
- [52] Schumock, G., et al., Integrated Risk-Informed Design (I-RID) Methodological Framework and Computational Application for FLEX equipment Storage Buildings of Nuclear Power Plants, Progress in Nuclear Energy, 120 (2020), 103186
- [53] Kichline, M., et al., Use of Expert Judgment to Support Human Reliability Analysis of Implementing FLEX Equipment, *Proceeding*, 14th International Topical Meeting on Probabilistic Safety Assessment and Management (PSAM14), Los Angeles, Cal., 2018
- [54] Montecalvo, M., Lessons Learned from Modeling the FLEX Steam Generator Feed Pump for Palo Verde Nuclear Generating Station. U.S. Nuclear Regulatory Commission (2017), ML17128A452, May 2017
- [55] Dermarkar, F., et al., Guidance on the Implementation of Modifications to Mitigate Beyond Design Basis Accidents, International Conference on Topical Issues in Nuclear Installation Safety: Defence in Depth – Advances and Challenges for Nuclear Installation Safety, *Proceedings*, International Conference Held in Vienna, Austria, December 11-15., 21-24 October 2013, IAEA-TECDOC-CD-1749, pp. 88-95
- [56] Dermarkar, F., Human and Organizational Considerations in Severe Accidentmanagement, *Proceedings*, IAEA International Experts' Meeting on Severe Accident Management in the Light of the Accident at the Fukushima Daiichi Nuclear PowerPlant, International Atomic Energy Agency, Vienna, Austria, 2014
- [57] ***, Canadian Nuclear Safety Commission, Design of New Nuclear Power Plants, Canadian Nuclear Safety Commission Regulatory Document (2014), REGDOC-2.5.2.
- [58] Viktorov, A., Harwood, C., Design Extension Conditions Concept and its Application to Operating Reactors in Canada, *International Nuclear Safety Journal*, 4 (2015), 3, pp. 13-23

- [59] Harter, R., Severe Accident Management and Beyond Design Bases Event Response-an end-user Perspective. A Presentation at the IAEA International Experts Meeting on Severe Accident Management in the Light of the Accident at the Fukushima-Daiichi Nuclear Power Plant, IAEA, Vienna, Austria, 2014
- [60] Vayssier, G., Present Day EOPS and SAMG Where Do We Go from Here? *Nuclear Engineering and Technology*, 44 (2012), 3, pp. 225-236
- [61] Lavarenne, C., et al., How to Reinforce the "Defence-in-Depth" in NPP by Taking into Account Natural Hazards?. Proceedings, International Conference on Topical Issues in Nuclear Installation Safety: Defence in Depth – Advances and Challenges for Nuclear Installation Safety (2013), IAEA-TECDOC-CD-1749, pp. 56-65
- [62] Lizot, M. T., et al., Post-Fukushima Complementary Safety Assessment Outcomes for French Fuel Cycle Facilities, Progress in Nuclear Energy, 84 (2015), pp. 97-102
- [63] Lavarenne, C., Main Conclusions of the French NPP Stress Tests, 'A Need for a Hardened Safety Core'. Proceedings, IAEA International Experts Meeting on Reactor and Spent Fuel Safety in the Light of the Accident at the Fukushima Daiichi NPP, IAEA Vienna, 2012
- [64] Mariscal, M. A., et al., Assessing Safety Culture in the Spanish Nuclear Industry Through the Use of Working Groups, Safety Science, 50 (2012), 5, pp. 1237-1246
- [65] Chung, W. S., et al., Renovated Korean Nuclear Safety and Security System: A Review and Suggestions to Successful Settlement, *Proceedings*, 2012 International Congress on Advances in Nuclear Power Plants – ICAPP '12, Chicago, Ill., USA, 2012, pp. 1433-1437
- [66] Lee, S. W., et al., Comparative Analyses on Containment Over-Pressurization Mitigation Strategy in PWR Using MAAP Codes, IAEA-TECDOC-1812, *Proceedings*, Technical Meeting on Severe Accident Mitigation Through Improvements in Filtered Containment Venting for Water Cooled Reactors, Vienna, Austria, August, 2015, pp. 112-118,
- [67] Na, J. H., KHNP's Strategies for Multi-Unit Extreme Hazards Response, Safety Technology Center Central Research Institute, KHNP, 2018

- [68] Kim, D. H., et al., Development of Mitigation Strategy for Beyond Design Basis External Events for NRC Design Certification, *Proceedings*, Transactions of the Korean Nuclear Society Autumn Meeting, Gyeongju, Korea, 2013
- [69] Kim, S. H., et al., A Passive Decay Heat Removal Strategy of the Integrated Passive Safety System (IPSS) for SBO Combined with LOCA, Nuclear Engineering and Design, 295 (2015), pp. 346-359
- [70] Jang, K. B., Woo, T. H., Analysis of Humanoid Robotics for Nuclear Disaster Management Incorporated with Biomechanics, *Nucl Technol Radiat*, 34 (2019), 3, pp. 291-298
- [71] Xu, H., et al., Design and Preliminary Analysis of In-Vessel Core Catcher Made of High-Temperature Ceramics Material in PWR, Atomic Energy Science and Technology, 45 (2011), 11, pp. 1334-1339
- [72] Xu, H., Zhou, Z., Research on Uncertainty Analysis of In-Vessel Retention Parameters in Advanced PWR, *Atomic Energy Science and Technology*, 46 (2012), 1, pp. 37-42
- [73] Xu, H., Study on Mitigation Strategy of AP1000 Spent Fuel Pool Exterior Disaster, *Atomic Energy Sci*ence and Technology, 46 (2012), Suppl., pp. 473-478
- [74] Zheng, L. X., et al., Study on U.S. NRC's FLEX Strategies in Cope with Beyond Design-Basis External Events after Fukushima Accident, *Nuclear Safety*, 15 (2016), 5, pp. 34-39
- [75] Xu, H., Li, G., Research on Key Technical Requirements for the FLEX Strategy Development, *Proceedings*, Conference of NPIC FLEX Project, Chengdu, China, 2020
- [76] Xu, H., et al., New Safety Strategies for Nuclear Power Plants: a Review, International Journal of Energy Research, 45 (2021), 8, pp. 11564-11588
- [77] Yu, Y., et al., An Integrated EDMG to Deal With Extensive Damage for NPP in China, *Proceedings*, 2017 25th International Conference on Nuclear Engineering (ICONE25), Shanghai, China, 2017

Received on January 28, 2021 Accepted on April 4, 2021

Хунг СЈУ, Баожуеј ЏАНГ

ПРЕГЛЕД КОНЦЕПТА ОДБРАНЕ ПО ДУБИНИ И FLEX СТРАТЕГИЈА У РАЗЛИЧИТИМ ЗЕМЉАМА НАКОН НЕСРЕЋЕ У ФУКУШИМИ

Да би побољшао одбрану по дубини нуклеарне сигурности након несреће у Фукушими, Амерички институт за нуклеарну енергију изнео је концепт разноврсних и прилагодљивих стратегија деловања и одговарајуће FLEX смернице подршке за посебне сценарије продуженог губитка наизменичне струје и крајњег губитка снаге хладиоца узроковане спољашњим догађајем изван пројектних основа. Након тога, идеја FLEX стратегије широко је прихваћена и распростањена. Увођење концепта FLEX стратеије у одбрану по дубини било је највеће побољшање нуклеарне сигурности у последњој деценији. У раду је детаљно размотрен концепт традиционалне одбране по дубини и његове слабости које су довеле до нуклеарне несреће у Фукушими, што је изазвало пораст мотивације за развој FLEX стратегије. Приказан је преглед напретка истграживања FLEX стратегије у различитим земљама у последњих десет година. На основу литературе и наведеног прегледа представљене су неке препорукуе за будући рад.

Кључне речи: FLEX сшрашегија, одбрана йо дубини, сйољашњи догађај изван йројекшних основа, йродужени губишак снаге наизменичне сшрује, крајњи губишак снаге хладиоца, нуклеарни акциденш у Фукушими